

South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

January 30, 2012 NOC-AE-12002791 File No.: G25

10 CFR 50.73 STI: 33280371

U. S. Nuclear Regulatory Commission Attention: Document Control Desk One White Flint North 11555 Rockville Pike Rockville, MD 20852-2738

South Texas Project
Unit 2
Docket No. STN 50-499
Licensee Event Report 2-2011-002
Unit 2 Reactor Trip on Main Generator Lockout

Pursuant to 10 CFR 50.73, STP Nuclear Operating Company (STPNOC) submits the attached Unit 2 Licensee Event Report (LER) 2-2011-002 to address the Unit 2 Reactor trip that occurred on November 29, 2011.

This condition is considered reportable under 10 CFR 50.73(a)(2)(iv)(A), any event or condition that resulted in manual or automatic actuation of any of the systems listed in paragraph (a)(2)(iv)(B) of this section.

This event did not have an adverse effect on the health and safety of the public.

There are no commitments contained in this LER. Corrective actions will be implemented in accordance with the STP Corrective Action Program.

If there are any questions on this submittal, please contact either Jamie Paul at (361) 972-7344 or me at (361) 972-7566.

G. T. Powell VP Generation

JLP

Attachment: LER 2-2011-002

CC:

(paper copy)

Regional Administrator, Region IV U. S. Nuclear Regulatory Commission 1600 East Lamar Blvd Arlington, Texas 76011-4511

Balwant K. Singal Senior Project Manager U.S. Nuclear Regulatory Commission One White Flint North (MS 8B1) 11555 Rockville Pike Rockville, MD 20852

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 289, Mail Code: MN116
Wadsworth, TX 77483

C. M. Canady
City of Austin
Electric Utility Department
721 Barton Springs Road
Austin, TX 78704

(electronic copy)

A. H. Gutterman, Esquire Morgan, Lewis & Bockius LLP

Peter Nemeth Crain Caton & James, P.C.

John Ragan Chris O'Hara Jim von Suskil NRG South Texas LP

Kevin Pollo Richard Pena City Public Service

C. Mele City of Austin

Richard A. Ratliff
Texas Department of State Health
Services

Alice Rogers Texas Department of State Health Services

Balwant K. Singal U. S. Nuclear Regulatory Commission

U.S. NUCLEAR REGULATORY COMMISSION (10-2010) LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block) 1. FACILITY NAME South Texas Unit 2				APPROVED BY OMB: NO, 3150-0104 Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53). U.S. Nuclear Regulatory Commission, Washington. DC 20555-0001, or by internet e-mail to Infocollects.resource@nrc.gov , and to the Oesk Officer, Office of Information and Regulatory Affairs. NEOB-10202. (3150-1104), Office of Management and Budget. Washington. DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to the information collection. 2. DOCKET NUMBER 3. PAGE					
4. TITLE	Unit 2 Rea	actor Tri							
5. EVENT DATE	6. LER NUMBER		7. REPO	ORT DAT	E	8. OTHER FAC	ILITIES INVO	LVED	
MONTH DAY YEAR	YEAR SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAM	ITY NAME DOCKET NUME		NUMBER
11 29 2011	2011 002	0	01	30	2012	N/A N/A		N/A N/A	
	2011 002	<u> </u>	UI	30	2012	19/7		11//	
9. OPERATING MODE	11. THIS REPORT IS SUBM	IITTED PUR	SUANT TO	THE RE	QUIREMEN	ITS OF 10CFR§:	(Check all tha	t apply)	
1 10. POWER LEVEL 100%	20.2203(a)(2)(i) 50.36(c)(1)(i)(A)				□ 50.73(a)(2)(i)(C) □ 50.73(a)(2)(vii) □ 50.73(a)(2)(ii)(A) □ 50.73(a)(2)(viii)(A) □ 50.73(a)(2)(ii)(B) □ 50.73(a)(2)(viii)(B) □ 50.73(a)(2)(iii) □ 50.73(a)(2)(ix)(A) □ 50.73(a)(2)(iv)(A) □ 50.73(a)(2)(x)				
	□ 20.2203(a)(2)(iii) □ 50.36(c)(2) □ 20.2203(a)(2)(iv) □ 50.46(a)(3)(ii) □ 20.2203(a)(2)(v) □ 50.73(a)(2)(i)(A) □ 20.2203(a)(2)(vi) □ 50.73(a)(2)(i)(B)			□ 50.73(a)(2)(v)(A) □ 73.71(a)(4) □ 50.73(a)(2)(v)(B) □ 73.71(a)(5) □ 50.73(a)(2)(v)(C) □ OTHER □ 50.73(a)(2)(v)(D) Specify in Abstract below or in NRC Form 366A				ow	
FACILITY NAME	12. LI	ICENSEE C	ONTACT F	OR THIS	LER		TELEPHONE NU	MRER (Includ	te Area Code)
Jamie Paul, Licensing	Engineer						361-972-7		ic Area Gode)
13. CO	MPLETE ONE LINE FOR EACH	COMPON	ENT FAILUF	RE DESC	RIBED IN 1	THIS REPORT			
CAUSE S YSTEM COMPO		REPORTABLI EPIX	E TO CAU	SE S	YSTEM	COMPONENT	MANU- FACTURER	REPOR	RTABLE TO EPIX
14. SUPF	PLEMENTAL RESPONSE EXP		NO		15. EXPEC SUBMISSI DATE		MONTH	DAY	YEAR
ABSTRACT (Limit to 1400 space On 11/26/11 at 2121 hour generator thermocouple T conducting a local check s 11/27 multiple Generator thermocouple from service Unit 2 Reactor tripped at 0 significant stator coil dama. The failure analysis determined the significant stator and signif	is, Unit 2 received the Sta 6144 on the SCW outlet subsequently reported the Condition Monitoring (GC e by substituting a known 0329 hours on 11/29/11 d age. Approximately three mined the most likely cau	ator Coil V of Coil 33 e Coil 33 CM) alarm n value. Al due to Ma rete of st use was a o travel in	Vater (SC BT was rea If thermood is were reat 0310 on in General actor Coil in very small side the co	W) Differed by Dif	igher tha vas readi . The ope 11 a Stat ckout. An p coil in s in a hollo e moistur	n the other the ng within the erating crew sor Cooling Winitial inspects lot 33) was now strand in Coe degraded the	differential ubsequentlater trouble tion of the related or mine tesin in the resin in the resin in the difference of the resin in the difference of the resin in the difference of the difference of the resin in the difference of	es. I&C T temperat y remove a alarm wa nain gene ssing on alysis sup	echnicians ure band. On the the Coil 33T as received. The trator revealed the exciter end. poorts that this lowing the

The failure analysis determined the most likely cause was a very small leak in a hollow strand in Coil 33T. Analysis supports that this leak existed for a long time and allowed moisture to travel inside the coil. The moisture degraded the resin in the coil allowing the conductor bundle to come loose. This condition allowed some individual conductor strands to move and vibrate. The strand-to-strand vibrations were away the insulation and created shorts. The shorts caused excessive heating. The affected area grew due to thermal damage until the coil arced across the missing melted area. The potential exists that the small leak was located in the portion of the coil that is melted/missing. If so, it will not be possible to ascertain the root cause of the leak. After the final failure analysis reports are received, the root cause report and LER will be evaluated and revised if warranted.

This condition is considered reportable under 10 CFR 50.73(a)(2)(iv)(A), any event or condition that resulted in manual or automatic actuation of any of the systems listed in paragraph 10 CFR 50.73(a)(2)(iv)(B). There were no personnel injuries, no offsite radiological releases, and no damage to safety-related equipment associated with this condition. This condition did not have an adverse effect on the health and safety of the public.

NRC FORM 366A LICENSEE EVENT REPORT (LER) U.S. NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET							
1. FACILITY NAME	ILITY NAME 2. DOCKET 6. LER NUMBER						
South Texas Unit 2	05000499	YEAR	SEQUENTIAL NUMBER	REV. NO	2 OF 5		
Oddii Texas Offic 2	03000499	2011	002	00			

I. DESCRIPTION OF EVENT

A. REPORTABLE EVENT CLASSIFICATION

This event is reportable pursuant to 10 CFR 50.73(a)(2)(iv)(A), any event or condition that resulted in manual or automatic actuation of any of the systems listed in paragraph (a)(2)(iv)(B) of this section.

B. PLANT OPERATING CONDITIONS PRIOR TO EVENT

South Texas Project (STP) Unit 2 was in Mode 1, with Reactor Power at approximately 100%.

C. STATUS OF STRUCTURES, SYSTEMS, AND COMPONENTS THAT WERE INOPERABLE AT THE START OF THE EVENT AND THAT CONTRIBUTED TO THE EVENT

No structures, systems, or components were inoperable at the start of the event that contributed to the event.

D. NARRATIVE SUMMARY OF THE EVENT

On 11/26/11 at 2121 hours, Unit 2 received the Stator Coil Water (SCW) Differential Temperature high alarm. The crew responded by implementing 0POP09-AN-7M03 for the A-5 annunciator. The crew identified a differential temperature greater than 14.4 °F on the Integrated Plant Computer System (ICS). None of the ICS thermocouple points indicated greater than the 174 °F criteria for notifying the System Engineer. The crew determined generator thermocouple T6144 on the SCW outlet of Coil 33T was reading higher than the other thermocouples. The crew contacted the Integrated Maintenance Team for I&C support to verify the Coil 33T thermocouple reading in accordance with the annunciator response procedure.

On 11/27/11 at approximately 0200 hours, two I&C Technicians and their Supervisor arrived at the Generator Terminal Board at the east side of the Main Generator. The I&C Technicians reported to the Unit 2 Control Room that the Coil 33T thermocouple was reading 160.7 °F. Shortly after the I&C Technicians disconnected their equipment at approximately 0222 hours, thermocouple 33T rose to 168 °F and it continued rise unnoticed until it read greater than 175 °F. The crew mistakenly believed the higher temperature indicated a malfunctioning thermocouple due to I&C activities based on the report that the Coil 33T thermocouple was reading 160.7 °F at the generator. At 0235 hours, the Coil 33T thermocouple rose to 180.35 °F but the crew was not trending or monitoring the point because it was considered an invalid indicator.

At 0244 hours, the Control Room received multiple Generator Condition Monitor (GCM) alarms from the ICS. The Control Room directed a Plant Operator (PO) to adjust the analyzer's sample flow. Based on the behavior of the GCM alarms and other indicators, the crew determined the GCM was malfunctioning and a Condition Report (CR) was written on the GCM. At 0254 hours, the GCM Verified Alarm cleared. At 0342 hours, the crew removed the Coil 33T thermocouple from service by substituting a known value for the thermocouple data. When the Coil 33T thermocouple was removed from service it had trended down from 181.7 °F to 178.7 °F.

NRC FORM 366A LICENSEE EVENT REPORT (LER) U.S. NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET							
1. FACILITY NAME	2. DOCKET	6. LE	RNUMBER	3. PAGE			
South Texas Unit 2	05000499	YEAR	SEQUENTIAL NUMBER	REV. NO	3 OF 5		
	05000499	2011	002	00			

On 11/29/11 at 0216 hours, the GCM Warning Alarm began to cycle in and out. The Control Room was focused on other activities and believed the GCM was malfunctioning. Between 0217 and 0241 hours the GCM generated 105 GCM Warning and 3 GCM Verified Alarms. The Control Room dispatched a PO to check the GCM. The PO notified the Control Room that the GCM was erratic and cycling between safe and alarm. The PO left the GCM to obtain a copy of procedure 0POP02-GG-0001, Generator Hydrogen and Carbon Dioxide Gas System, Addendum 4, Generator Condition Monitoring Alarm Response to address the GCM alarms.

At 0310 hours, the Control Room received annunciator 7M03-A6, Stator Cooling System Trouble Alarm. The Control Room redirected the PO to check the Stator Cooling Water System. When the PO arrived at the Stator Cooling Water skid, he noted that the tank level was high and water conductivity was rising. The PO immediately contacted Chemistry to sample the cooling water for conductivity.

The Unit 2 Reactor tripped at 0329 hours on 11/29/11 due to Main Generator Lockout. A "crawl through" inspection was performed on the day after the Unit 2 Main Generator event and reactor trip and significant stator coil damage was found. Approximately three feet of stator Coil 33T (i.e., the Top coil in slot 33) was melted or missing on the exciter end.

The coils have hollow strands throughout the coil to allow Stator Cooling Water (SCW) to flow through the coils themselves. Each strand is covered with resin used as internal fillers and insulation inside the coil. The insulation surrounding the conducting portion of the coil is called groundwall insulation and it consists of mica covered glass backed tape. The resin bonds the mica tape in the groundwall and fill voids between the groundwall and the conductor stack.

The failure analysis determined the most likely cause was that a very small leak existed in a hollow strand in Coil 33T. Analysis supports that this leak existed for a long time and allowed moisture to travel inside the coil. The moisture degraded the resin in the coil over time allowing the conductor bundle to come loose from the ground wall and allow some individual conductor strands to move and vibrate. The strand-to-strand vibrations and movement eventually wore away the insulation between the strands and created strand-to-strand shorts. The shorts caused excessive heating. The affected area grew as the coil insulation was thermally damaged and failed. As the affected area grew, more heat was created until the coil melted and eventually arced violently across the missing melted area eventually causing the 33T coil to catastrophically fail.

The leak in a hollow strand wall could have been caused by several different mechanisms. Inspections did not identify the leak location. The exact cause of the leak cannot be ascertained without being able to examine the leak location. Although the final analysis report is not yet available, it is likely that the small leak was located in the portion of the coil that is melted/missing. If so it will not be possible to identify the root cause of the leak. After the final failure analysis reports are received, the root cause report will be evaluated and revised if warranted based upon the findings. The LER will be supplemented if the evaluation results affect the substance of this report.

The failure of 33T coil itself did not cause the ground fault relay due to its position to neutral (first coil from neutral) and its low voltage, but when 33T failed, melted copper was expelled from the ground wall of 33T and flowed down on 33B. The melted copper degraded the groundwall insulation on 33B and a ground fault occurred.

NRC FORM 366A LICENSEE EVENT REPORT (LER) U.S. NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET							
1. FACILITY NAME	2. DOCKET	6. LE	3. PAGE				
South Texas Unit 2	05000499	YEAR	SEQUENTIAL NUMBER	REV. NO	4 OF 5		
South Texas Offic 2	03000499	2011	002	00	·		

E. METHOD OF DISCOVERY

The generator failure, reactor trip, and automatic actuation of the systems listed below were self-revealing.

II. EVENT-DRIVEN INFORMATION

A. SAFETY SYSTEMS THAT RESPONDED

All required safety systems responded as expected including the following actuations:

- 1. Reactor Coolant Pump Undervoltage Reactor Trip
- 2. Reactor Protection System P-16, Turbine Trip
- 3. Feedwater Isolation Actuation
- 4. CRE HVAC Emergency Recirculation (C Train LOOP)
- 5. Reactor Containment Fan Coolers (C Train LOOP)
- 6. Auxiliary Feedwater Actuation (All AFW pumps actuated)
- 7. Primary Pressure Control (Pressurizer Spray and Heaters actuated as required)
- 8. Secondary Pressure Control Actuation (Steam Dumps Actuated)

B. DURATION OF SAFETY SYSTEM INOPERABILITY

N/A

C. SAFETY CONSEQUENCES AND IMPLICATIONS OF THE EVENT

There was no impact to radiological safety, safety of the public, or safety of station personnel during this event.

The Incremental Conditional Core Damage Probability (ICCDP) for the Reactor Trip in Unit 2 on November 29, 2011 is 2.65E-07. The resulting Incremental Conditional Large Early Release Probability (LERP) is 5.99E-09.

III. CAUSE OF THE EVENT

Although the final analysis report is not yet available, it is likely that the small leak was located in the portion of the coil that is melted/missing. If so the root cause of the leak will not be determined. After the final failure analysis reports from ElectroMechanical Engineering (EME) and Kinectrics are received, the root cause report will be revised and the corrective action plan addressing the technical aspects of this event will be changed if warranted based upon the findings. The LER will be supplemented if the evaluation results affect the substance of this report.

NRC FORM 366A LICENSEE EVENT REPORT (LER) U.S. NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET								
1. FACILITY NAME	2. DOCKET	6. LE	RNUMBER	3. PAGE				
South Texas Unit 2	05000499	YEAR	SEQUENTIAL NUMBER	REV. NO	5 OF 5			
Godin Toxas Offic 2	05000439	2011 002 00		00				

IV. CORRECTIVE ACTIONS

Corrective actions will be implemented in accordance with the STP Corrective Action Program.

Enhancement actions are planned to improve control room annunciation and indication of generator conditions.

Repairs to the Unit 2 Main Generator stator, rotor, exciter, hydrogen cooler, and associated auxiliary equipment are in progress.

As discussed above, following receipt of the final failure analysis reports, the root cause report will be revised and the corrective action plan addressing the technical aspects of this event will be changed if warranted based upon the findings.

V. PREVIOUS SIMILAR EVENTS

There have been no similar reportable events at STP within the last three years.

VI. ADDITIONAL INFORMATION

N/A